



35 U.S. PTO PATENT APPLICATION TRANSMITTAL <small>(Only for new nonprovisional applications under 37 CFR 1.53(b))</small>	Attorney Docket No.	198801US3
	First Inventor or Application Identifier	Kiichiro SAKASHITA, et al.
	Title AN ABSORBING ROD, AN APPARATUS FOR INSERTING THE ABSORBING ROD, A CASK, AND A METHOD OF STORING SPENT FUEL ASSEMBLIES	

APPLICATION ELEMENTS <small>See MPEP chapter 600 concerning utility patent application contents</small>	ADDRESS TO: Assistant Commissioner for Patents Box Patent Application Washington, DC 20231
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18. CORRESPONDENCE ADDRESS	
 22850 (703) 413-3000 FACSIMILE: (703) 413-2220	

Name:	C. Irvin McClelland	Registration No.:	21,124
Signature:		Date:	10/27/00
Name:		Registration No.:	

Docket No. 198801US3

IN THE UNITED STATES PATENT AND TRADEMARK OFFICE

INVENTOR(S) Kiichiro SAKASHITA, et al.

SERIAL NO: New Application

FILING DATE: Herewith

FOR: AN ABSORBING ROD, AN APPARATUS FOR INSERTING THE ABSORBING ROD, A CASK, AND A METHOD OF STORING SPENT FUEL ASSEMBLIES

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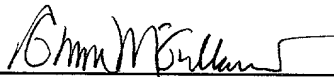
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Respectfully Submitted,

OBLON, SPIVAK, McCLELLAND,
MAIER & NEUSTADT, P.C.



C. Irvin McClelland

Registration No. 21,124

Date: 10/27/00



22850

Tel. (703) 413-3000
Fax. (703) 413-2220
(OSMMN 10/00)

AN ABSORBING ROD,
AN APPARATUS FOR INSERTING THE ABSORBING ROD, A CASK,
AND A METHOD OF STORING SPENT FUEL ASSEMBLIES

5 FIELD OF THE INVENTION

 The present invention relates to an absorbing rod
having a neutron absorbing capability to be inserted into
a control rod guide pipe of spent fuel assemblies, an
inserting apparatus for inserting the absorbing rod into the
10 control rod guide pipe, a cask for containing, conveying and
storing the spent fuel assemblies, and a conveying and
storing method of spent fuel assemblies.

BACKGROUND OF THE INVENTION

15 Fuel assemblies consumed and no longer usable in a
terminal stage of fuel cycle are called spent fuels. Spent
fuels contain fission products and other highly radioactive
substances, and must be cooled thermally, and hence they are
cooled for a specific period in a cooling pit at a nuclear
20 power plant. Then they are transferred into a shielding
container called cask, and conveyed and stored at
reprocessing plant or storage facility by means of truck or
ship. When transferring the spent fuel assemblies into the
cask, a holding element having a lattice section called
25 basket is used. Each one of the spent fuel assemblies is

put into each of the cells formed as plural storing spaces in the basket, and a proper holding strength to withstand vibration during transportation is assured.

Various types of casks are disclosed in publications such as "Atomic Eye" (Nikkan Kogyo Shuppan Production, April 1, 1998) and Japanese Patent Application Laid-open No. 62-242725.

Fig. 13 is a perspective view of an example of a cask. Fig. 14 is an axial direction sectional view of the cask shown in Fig. 13. A cask 500 is composed of a shell main body 501, a resin 502 which is a neutron shield provided on the outer circumference of the shell main body 501, its outer tube 503, bottom 504 and lid 505. The shell main body 501 and bottom 504 are carbon steel forged parts which are gamma-ray shielding materials. The lid 505 is composed of a primary lid 506 and a secondary lid 507 made of stainless steel or the like. The shell main body 501 and bottom 504 are bonded by butt welding. The primary lid 506 and secondary lid 507 are fixed to the shell main body 501 by means of stainless steel bolts. A metal O-ring is interposed between the lid 505 and shell main body 501, and the inside is kept airtight.

Many inner fins 508 are provided, between the shell main body 501 and outer tube 503, for heat conduction. The inner fins 508 are made of copper in order to enhance the heat conduction efficiency. The resin 502 is injected into

the space formed by the inner fins 508 in fluid state, and is solidified by thermosetting reaction or the like. A basket 509 is a bundle structure of 69 square pipes 510 as shown in Fig. 14, and is inserted into a cavity 511 in the shell main body 501.

The square pipes 510 are made of aluminum alloy mixing neutron absorbing material (boron: B) so that the inserted spent fuel assemblies may not reach the criticality. At both sides of the cask main body 512, moreover, a trunnion 513 (only one is shown) for suspending the cask 500. At both ends of the cask main body 512, a buffer materials 514 assembling wood or other shock-absorbing materials are provided (only one side is shown). Reference numeral 515 is a cell for accommodating the spent fuel assemblies.

The light-water type reactor is classified into the boiling water reactor (BWR) and pressurized water reactor (PWR). This cask 500 is the cask for storing the spent fuel assemblies used in BWR. The structure of the fuel assembly for BWR is shown in Fig. 15. The fuel assembly for BWR has four channel boxes 601 arranged in lattice, and a cross-shaped control rod 608 is located in the central area, and the reactivity of fuel rods 603 is controlled by the vertical move of the control rod 608.

The fuel assembly 600 has 8×8 fuel rods 603 arranged in lattice. Each fuel rod 603 is a long bar of about 4 meters.

1
A support lattice 607 is provided for supporting the fuel
rod 603. In each fuel rod 603, plural columnar pellets 631
formed by sintering powder of uranium oxide are inserted as
fuel into a cylindrical clad pipe 633 composed of zirconium
5 alloy, and held by a spring 632 inserted in the upper part
of the clad pipe 633. A handle 610 is used for lifting up
and down the fuel assembly 600. The spent fuel assembly
inserted in the cell 515 in this cask 500 is a fuel assembly
for BWR 600 shown in Fig. 15.

10 By contrast, the fuel assembly shown in Fig. 16 is used
in PWR. In this fuel assembly for PWR 700, control rods 708
are properly distributed among fuel rods 703, and the fuel
rods 703 and control rods 708 are arranged in 17×17 lattice
form. In this 17×17 fuel assembly 700, one measuring pipe
15 is disposed in the center, and 24 control rods 708 are
distributed.

The reactivity of the core can be controlled by moving
the control rod 708 up or down. Each fuel rod 703 is a long
bar of about 4 meters same as the fuel rod 603 for BWR, and
20 hence a support lattice 707 for supporting the fuel rod 703
is provided. In each fuel rod 703, plural pellets 731 are
inserted into a cylindrical clad pipe 733 composed of
zirconium alloy, and held by a spring 732 inserted in the
upper part of the clad pipe 733. In this cask, 70 units of
25 spent fuel assemblies are contained in the cell.

Fig. 17 is an axial direction sectional view of the cask for PWR. As shown in Fig. 17, a cavity 811 of a cask 800 for PWR contains a basket 809 forming a cell 815 of rectangular section by alternately combining plates 817 extending in the radial direction. Each plate 817 is composed of aluminum alloy mixing B as neutron absorbing material, same as the square pipes 510 for BWR.

However, each plate 817 has a through-hole of cooling water passage 816 extending in the axial direction, known as water zone, and when cooling the spent fuel assemblies, each basket cell and this through-hole are filled with water to moderate neutrons, so that the neutrons may be absorbed efficiently by the plates 817 and the resin 802. The water filling the cooling water passage 816 is discharged after a specified cooling period, and the passage is dried.

This cooling water passage 816 is required because the uranium enrichment factor of fuel assembly for PWR is higher than that of the fuel assembly for BWR, the uranium charging amount per fuel assembly is larger, the sectional area of the fuel assembly is wider, and hence the reactivity is higher in the system arranging these assemblies. The distance dd shown in Fig. 17 is such a distance that it assures the subcriticality of the spent fuel assembly. The distance dd for PWR is required to be longer than the same for BWR. The cells 815 are not arranged in lattice but are deviated in

position, which is intended to dispose the spent fuel assemblies for PWR, having a wider sectional area as compared with the spent fuel assemblies for BWR, efficiently in the cavity 811.

5 In the spent fuel assemblies for PWR mentioned above, the control rods 708 distributed among the fuel rods 703 arranged in lattice are extracted, and the spent fuel assemblies 700 from which the control rods 708 are extracted are inserted and stored in the cells 815 formed by the basket
10 809 in the cask 800.

Therefore, the vacancies left over by the extracted control rods 708 or measuring rod, not shown, in the spent fuel assemblies are dead spaces, and it is desired to utilize these spaces effectively.

15 Further, the plates or square pipes for forming the basket for PWR are required to form the cooling water passage 816, and formation of plates or square pipes is more complicated as compared with formation of plates or square pipes for BWR, and it takes much time and labor.

20 Besides, the sectional area of the spent fuel assemblies for PWR is rectangular and wide, and also the cooling water passage 816 is needed, and hence the accommodation efficiency of the spent fuels in the cask is poor.

25

SUMMARY OF THE INVENTION

It is an object of the present invention to provide an absorbing rod capable of heightening the accommodation density of spent fuel assemblies, by effectively utilizing the control rod inserting space and measuring rod inserting space existing in the spent fuel assemblies for PWR and shortening the distance dd for assuring the subcriticality among the spent fuel assemblies, its inserting apparatus, cask, and conveying and storing method of spent fuel assemblies.

The absorbing rod according to one aspect of this invention has nearly the same shape as the shape of a columnar control rod for PWR. Further, the absorbing rod has the ability to shield the neutrons. Since the absorbing rod has nearly the same shape as the shape of the columnar control rod for PWR, this absorbing rod can be inserted into the control rod guide pipe of spent fuel assemblies or the control rod guide pipe including the measuring pipe.

The absorbing rod according to another aspect of this invention is made of aluminum composite material or aluminum alloy formed by adding powder of boron or boron compound having a neutron absorbing performance to aluminum or aluminum alloy powder. Since the absorbing rod is made of such material, it does not get damaged by minor shocks during transportation. As a consequence, it is possible to

maintain the neutron absorbing ability even during transport. For example, it is much safer and therefore preferable than an absorbing rod having a structure in which boron powder is filled inside a pipe.

5 The cask according to still another aspect of this invention comprises an absorbing rod group having nearly the same shape as the shape of a columnar control rod for PWR. Further, the absorbing rod group has the ability to shield the neutrons. The spent fuel assemblies is stored in the
10 cask. Accordingly, the number of neutrons radiated from the spent fuel assemblies can be decreased. Further, the spacing between the spent fuel assemblies can be shortened.

 The inserting apparatus according to still another aspect of this invention comprises a position detection unit
15 which detects the distance between a through-hole group of the guide member and the control rod guide pipe group of the spent fuel assemblies, and their relative positions are adjusted accordingly. Further, the absorbing rod block is lowered so that the through-hole group acts as a guide hole,
20 and the long absorbing rod group can be inserted securely into the control rod guide pipe or the control rod guide pipe including the measuring pipe.

 The conveying and storing method of spent fuel assemblies according to still another aspect of this
25 invention comprises a step of inserting an absorbing rod

group having a nearly same shape as the shape of a columnar control rod for PWR used in reactivity control of core in a reactor, and having a neutron shielding capability, into a control rod guide pipe group including a measuring pipe of spent fuel assemblies for PWR. Further, the spent fuel assemblies for PWR are conveyed and stored in a state in which the absorbing rod group is inserted. Accordingly, the distance between spent fuel assemblies can be shortened.

Other objects and features of this invention will become apparent from the following description with reference to the accompanying drawings.

BRIEF DESCRIPTION OF THE DRAWINGS

Fig. 1 is a perspective view which shows the structure of a spent fuel assembly for PWR.

Fig. 2 is a diagram which shows the structure of an absorbing rod block to be inserted in the spent fuel assembly and also explains how the absorbing rod block inserted in the spent fuel assembly.

Fig. 3 is a plan of the spent fuel assembly.

Fig. 4 is a sectional view along line A-A of support lattice.

Fig. 5 is a diagram showing a structure of a fuel rod.

Fig. 6 is a radial direction sectional view which shows the structure of a cask according to the present invention.

Fig. 7 is a diagram which shows the structure of an inserting apparatus according to the present invention.

Fig. 8 is a sectional view along line B-B of a lower end plate.

5 Fig. 9A to Fig. 9C are examples of the photographed images.

Fig. 10 is a diagram which shows the inserting apparatus right after completion of insertion.

10 Fig. 11 is a diagram which shows the structure of spent fuel assembly after completion of insertion and a structure of a suspender.

Fig. 12 is a schematic diagram which shows storage configuration of spent fuel assemblies in a storage pool.

15 Fig. 13 is a perspective view showing a structure of a cask for BWR.

Fig. 14 is a radial direction sectional view which shows the structure of the cask shown in Fig. 13.

Fig. 15 is a diagram showing a structure of a fuel assembly for BWR.

20 Fig. 16 is a diagram showing a structure of a fuel assembly for PWR.

Fig. 17 is a radial direction sectional view which shows structure of a conventional cask for PWR.

25 DESCRIPTION OF THE PREFERRED EMBODIMENTS

Preferred embodiments of the absorbing rod, the inserting apparatus (apparatus for inserting the absorbing rod), the cask, and the method of conveying and storing the spent fuel assemblies according to the invention are explained in detail below while referring to the accompanied drawings. However, this invention is not limited only to these embodiments.

A spent fuel assembly conforming to the invention is explained in the first place. Fig. 1 is a perspective view showing a general view of a spent fuel assembly for PWR. As shown in Fig. 1, a fuel assembly 1, which is a spent fuel assembly, is formed as a lattice bundling of fuel rods 4, control rod guide pipes 5, and measuring pipes 6, being arranged in a matrix section of 14×14 . This matrix arrangement is supported by support lattices 7 provided at proper positions in the longitudinal direction of the fuel rods 4, control rod guide pipes 5, and measuring pipes 6. The control rod guide pipes 5 and measuring pipes 6 are supported by an upper nozzle 2 and a lower nozzle 3.

That is, the control rod guide pipes 5 and measuring pipes 6 are cylindrical in shape, and held by through-holes between the upper nozzle 2 and lower nozzle 3 provided corresponding to the matrix arrangement positions of the control rod guide pipes 5 and measuring pipes 6. In the control rod guide pipes 5, control rods having a neutron

absorbing capability in the reactor are inserted, and the reactivity of the core is controlled by the number and depth of insertion of control rods. The control rod guide pipes 5 are dispersed on the matrix arrangement. In the measuring pipes 6, measuring rods not shown in the drawing are inserted periodically in the reactor for measuring the temperature and radiation dose, and the information for core management is obtained. However, when stored as spent fuel assembly, the control rods inserted in the control rod guide pipes 5 are extracted, and similarly measuring rods are extracted from the measuring pipes 6.

Accordingly, as shown in Fig. 2, plural absorbing rods 11 having the same columnar shape as the control rods and measuring rods to be inserted in the control rod guide pipes 5 and measuring pipes 6 and having a neutron absorbing capability are newly prepared, and these absorbing rods are inserted into the control rod guide pipes 5 and measuring pipes 6 of the fuel assembly 1. One end of each absorbing rod 11 is bonded to an end plate 12 which is a rectangular plate member to be inserted inside from the upper opening of the upper nozzle 2, and the other end is formed in a taper, so as to be inserted easily into the control rod guide pipes 5 and measuring pipes 6.

The length of the absorbing rod 11 is long enough to envelop the effective length of the fuel. The radius of the

absorbing rod 11 is less than the inside diameter of the control rod guide pipe 5 and measuring pipe 6, and is preferred to be closer to the inside diameter of the control rod guide pipe 5 and measuring pipe 6 as far as possible, but a certain clearance is required considering the length of about 4 meters.

Fig. 3 is a plan of the fuel assembly 1 shown in Fig. 2. The upper nozzle 2 shown in Fig. 3 corresponds to the matrix arrangement, in which the control rod guide pipes 5 and measuring pipes 6 are disposed. There are plural through-holes for light water passages 21 in the reactor. The absorbing rod 11 is penetrating through the control rod guide pipe 5 and measuring pipe 6.

Fig. 4 is a sectional view along line A-A of support lattice 7 of the fuel assembly 1 shown in Fig. 2. The support lattice 7 supports the lattice state of 14×14 consisting of fuel rods 4, control rod guide pipes 5 and measuring pipes 6. The absorbing rods 11 penetrate through the control rod guide pipes 5 and measuring pipes 6 of the support lattice 7 shown in Fig. 4.

Therefore, the end plate 12 shown in Fig. 2 is bonded with the absorbing rod 11 so that the absorbing rod 11 may be held in the state inserted in the control rod guide pipe 5 and measuring pipe 6. By using an absorbing rod block 10 formed by bonding plural absorbing rods 11 and end plates 12,

a plurality of absorbing rods 11 can be inserted in batch into the control rod guide pipes 5 and measuring pipes 6. Alternatively, instead of bonding the absorbing rods 11 by using endplates 12, the absorbing rods 11 may be individually
5 inserted into the control rod guide pipes 5 and measuring pipes 6. However, when conveying the fuel assembly 1 in which the absorbing rods 11 are inserted, it is easier to hold the absorbing rods 11 by using the absorbing rod block 10.

10 Fig. 5 is a front view of the fuel rod 4. In Fig. 5, plural pellets 31 are inserted into a cylindrical clad pipe 33 composed of zirconium alloy, and the pellets 31 are held by a spring 32 inserted in the upper part of the clad pipe 33. This fuel pipe 4 is mainly composed of the spent fuel
15 assembly as mentioned above.

The absorbing rod 11 is composed of aluminum composite material or aluminum alloy formed by adding powder of boron or boron compound having neutron absorbing capability, to the powder of aluminum or aluminum alloy, and it is designed
20 not to reach the criticality from the inside of the spent fuel assembly.

The following material may preferably used as the aluminum composite material. In addition to adding the powder of boron or boron compound to the aluminum composite
25 material, third particles of oxides may be added.

Accordingly, mechanical alloying is performed and the strength of the material can be increased. To achieve this, powder of aluminum or aluminum alloy as the matrix material, boron or boron compound as the neutron absorber, and third
5 particles as oxides, nitrates, carbides, borides are mixed in the Attriter pan.

The amount of these third particles should preferably be greater than 0.1 weight percent and less than 30 weight percent. Such an amount is preferable because, if the amount
10 is less than 0.1 weight percent, then there is not much effect on strengthening the material. On the other hand, if the amount is greater than 30 weight percent, then the property of the material to expand decreases. Further, it is preferable that the diameters of the third particles in the
15 aluminum matrix is greater than 0.01 μm and less than 10 μm . The material having a diameter smaller than 0.01 μm is very costly and difficult to acquire. On the other hand, the material having a diameter greater than 10 μm does not contribute much to strengthening the material.

20 The amount of boron or boron compound should preferably be greater than 1 weight percent and less than 20 weight percent. Such an amount is preferable because, if the amount is less than 1 weight percent, then it does not absorb the neutrons properly. On the other hand, if the amount is
25 greater than 20 weight percent, then the property of the

material to expand decreases. Further, it is preferable that the diameters of the boron or boron compound is greater than 0.01 μm and less than 100 μm . The material having a diameter smaller than 0.01 μm is very costly. On the other hand, the material having a diameter greater than 100 μm degrades the strength.

Further, any material having a property to absorb the neutrons may be used instead of boron or boron compound. That is, Cd, Hf, or rare earth elements having bigger neutron absorbing cross section may be used. For example, boron or boron compound are used in case of BWR, however, Ag-In-Cd compound are used in case of PWR. In the Ag-In-Cd compound, it is general that In is 15 weight present and Cd is 5 weight present. Further, carbides such as Eu, Sm, Gd may be used.

Al_2O_3 , ZrO_2 , SiO_2 , MgO , TiO_2 , Cr_2O_3 may be used as the oxides in the third particles. Si_3N_4 , BN are the examples of nitrates that may be used. SiC, WC, Cr_2C_3 are the examples of carbides, and ZrBr_2 is the example of boride that may be used. Further, it is not necessary to use only the third particles mentioned here. Any material may be used according to the need.

Following material may be used as powder of aluminum or aluminum alloy. That is, pure aluminum (JIS 1xxx), Al-Cu aluminum compound (JIS 2xxx), Al-Mg aluminum compound (JIS 5xxx), Al-Mg-Si aluminum compound (JIS 6xxx), Al-Zn-Mg

aluminum compound (JIS 7xxx), Al-Fe aluminum compound (Fe content between 1 to 10 weight percent). Further, Al-Mn aluminum compound (JIS 3xxx) also may be used. The material to be used may be selected depending upon the required strength, expansion, processing ability, and temperature resistance.

A powder obtained by the rapid solidification has uniform and fine composition. Such a powder may be used in as powder of aluminum or aluminum alloy. Known methods such as unit roll method, double roll method, atomization methods such as air atomization or gas atomization may be used to obtain rapid solidification powders. It is preferable that the particles of the obtained powder of aluminum or aluminum alloy has a diameter of between 5 to 150 μm .

If the diameter is less than 5 μm , then the particles come together and form a mass. Further, if the diameter is less than 5 μm then it is difficult to use the atomization methods (it is required to separate the minute powder and the coast increases). On the other hand, if the diameter is greater than 150 μm , then again there is a limitation on the use of the atomization method, and it becomes difficult to mix the material with the minute addition particles. The most preferable diameter is 50 to 120 μm . The cooling speed should be above 102 degree centigrade per second. Most preferably, the cooling speed should be above 103 degree

centigrade per second.

On the other hand, boron or boron compound has a property that it has great capability to absorb fast neutrons. B_4C , B_2O_3 may be applied in the present invention. Out of these two, B_4C has more boron content per unit weight. Therefore, B_4C can absorb more neutrons with lesser quantity. Further, B_4C has more hardness, and therefore, preferable.

It is mentioned above that, the third particles are oxides, nitrates, carbides, borides. However, the particles other than these may be added. For, example, Zr or Ti may be added so as to increase the strength.

The material mixed in the above-mentioned manner and amount is subjected to mechanical alloying in Attriter mill. A rotating mill or a vibrating mill may be used instead of Attriter mill. When subjected to mechanical alloying, aluminum and aluminum compound is crushed by the balls and become flat. Further, boron or boron compound and the third particles ground to still finer size and they uniformly enter into the voids in the aluminum matrix. These flattened particles become particles of normal shape containing aluminum or aluminum compound and the third particles.

It is possible to increase the material strength greatly because of the addition of the third particles and subjecting to mechanical alloying. It was confirmed with an experiment that the material formed in this manner has

about three times higher material strength than the material formed using the ordinary mixers (for example, cross-rotary mixer or V mixer). Further, boron or born compound having high hardness get dispersed finely and uniformly inside the matrix so that the cohesion of boron can be prevented. Further, property to squeeze out can be increased greatly. Accordingly, dice for squeeze out is not so worn-out due to friction.

Further, when adding the third particles, aluminum or aluminum composite powder may be subjected to oxidation thereby creating a oxide layer on the surface of these particles, finely pulverize this oxide layer due to the mechanical alloying so as to cause the particles to disperse in the matrix. The oxidation of the aluminum or aluminum composite powder may be performed by heating in air. For example, aluminum or aluminum composite powder may be put into a furnace and agitated while being heated. When this process is carried out for a few hours, an oxide layer is formed on the surface of aluminum or aluminum compound.

Thus, the absorbing rods 11 is formed of aluminum composite material or aluminum alloy. Accordingly, it is light weight, has an ability to shield the neutrons, and the distance between the spent fuel assemblies can be decreased. In other words, if the same number of the spent fuel assemblies are to be used, then the cask becomes light-weight.

Any material that has the property to absorb the neutrons may be used instead of aluminum composite material or aluminum alloy. Accordingly, the fuel rod 4 may itself be used instead of the absorbing rod 11.

5 According to the first embodiment, since the absorbing rods 11 having neutron absorbing capability are inserted into the control rod guide pipes 5 and measuring pipes 6 of spent fuel assemblies, the neutrons can be absorbed by effectively utilizing the space in the control rod guide
10 pipes 5 and measuring pipes 6, and emission of neutrons from the inside of the spent fuel assemblies can be reduced, and therefore the spacing distance for subcriticality between spent fuel assemblies can be shortened. Or, by using the absorbing rod block 10 bonding plural absorbing rods 11 by
15 end plates 12, the absorbing rods 11 can be inserted in batch into the control rod guide pipes 5 and measuring pipes 6, so that the working efficiency is enhanced.

A second embodiment is explained below. The second embodiment relates to a cask for accommodating the spent fuel
20 assemblies in which absorbing rods 11 of the first embodiment are inserted, which eliminates the cooling water passage provided in the plates of the basket in this cask.

Fig. 6 is a radial direction sectional view of the cask, which is an improvement of the conventional cask for PWR shown
25 in Fig. 17. The plate 817 shown in Fig. 17 is high in the

uranium enrichment factor in the spent fuel assemblies for PWR, large in uranium charge amount per unit fuel assembly as compared with the fuel for BWR, and wide in the sectional area of fuel assembly, and hence the reactivity is high in the system arranging them, and therefore in order to absorb the neutrons efficiently by moderating the neutrons in cooling process, the cooling water passage 816 is filled with water as neutron moderator. The spent fuel assemblies in which the absorbing rods 11 explained in the first embodiment are inserted contain neutron absorbing materials in the spent fuel assemblies, and hence the neutrons released outside from the spent fuel assemblies are decreased.

As a result, it is possible to shorten the spacing distance for controlling with the subcriticality in cooling of spent fuel assemblies, and hence it does not require cooling water passage 816 in cooling process. Accordingly, the plates 50 for composing the basket 49 of the cask 40 for PWR shown in Fig. 6 do not have cooling water passage same as in the basket structure for BWR.

Hence, in the cask 40 for PWR shown in Fig. 6, although the capacity volume is same as in the cavity 811 of the cask 800 for PWR shown in Fig. 17, the thickness of the plates 50 can be reduced, so that more spent fuel assemblies can be accommodated. For example, in the cask 800 shown in Fig. 17, 19 spent fuel assemblies can be accommodated, whereas

in the cask 40 shown in Fig. 6, 32 spent fuel assemblies can be accommodated. By using the cask 40 in the second embodiment, 13 more spent fuel assemblies can be accommodated. In other words, the degree of integration of spent fuel assemblies can be heightened.

In the basket of the same shape as shown in Fig. 17, when the spent fuel assemblies in the first embodiment are accommodated, emission of neutrons is decreased, and hence the resin thickness can be reduced. As a result, it is possible to reduce the entire size of the cask when accommodating the same number of spent fuel assemblies.

In the second embodiment, the cell 55 is composed by combining the plates 50 in lattice form. However, the cell may be formed with square pipes.

Further, when accommodating the spent fuel assemblies in the first embodiment, the thickness of the plates for composing the basket of the cask can be reduced corresponding to the neutron absorbing capability of the absorbing rods inserted in the spent fuel assemblies, so that the number of spent fuel assemblies accommodated in the cask can be increased. Further, it is not necessary to install the cooling water passage, which is the feature of the cask for PWR, in the plates or square pipes, and plates or square pipes of simple structure as in the plates or square pipes for BWR can be used.

A third embodiment is explained below. In the first embodiment, by using the absorbing rod block 10 composed of plural absorbing rods 11, a plurality of absorbing rods 11 are inserted in batch to enhance the working efficiency. The
5 third embodiment is further intended to insert spent fuel assemblies of plural absorbing rods 11 more easily, safely and securely.

Fig. 7 shows the structure of the inserting apparatus according to the third embodiment. This inserting apparatus
10 comprises an upper end plate 61 coupling the absorbing rod 11, a lower end plate 62 for guiding the absorbing rod 11, and a suspender 64 for moving the upper end plate 61.

The upper end plate 61 corresponds to the absorbing rod block 10. The upper end plate 61 and lower end plate
15 62 are suspended by the suspender 63. Fig. 8 is a sectional view along line B-B of the lower end plate 62. The lower end plate 62 has a guide hole 77 for guiding the absorbing rod 11. The guide hole 77 is provided corresponding to the position of the control rod guide pipe 5 and measuring pipe
20 6. When the upper end plate 61 and lower end plate 62 are suspended by the suspender 63, when stopping still or in the initial state, the leading end of the absorbing rod 11 must be at least inserted into the guide hole 77. The width of the upper end plate 61 and lower end plate 62 in the horizontal
25 direction must be wide enough for allowing to be inserted

from the upper opening of the upper nozzle 2, and the width is required to be at least enough for forming the guide hole 77 of the control rod guide pipe 5 and measuring pipe 6.

A suspension hole 78 is provided almost in the center of the upper end plate 61. The inner wall of the suspension hole 78 has a dent 70 for hooking the claw 69 of the clamping tool 65 mentioned below. The clamping tool 65 rotates freely about the fulcrum 68 provided in a flat suspension plate 60 corresponding to the upper end plate 61, and it is provided in the upper part of the clamping tool 65, and the claw 69 provided in the lower part of the clamping tool 65 is always opened by a tension spring 66 for pulling the upper part of the clamping tool 65 mutually, so that the upper end plate 61 is held. When canceling holding of the upper end plate 61, that is, when separating the upper end plate 61 and lower end plate 62 from the suspension portion 64, the upper part of the clamping tool 65 is pushed open by overcoming the tensile force of the tension spring 66, by means of an air cylinder 67 for connecting the upper part of the clamping tool 65 mutually. As a result, the clamp 69 is released from the dent 70, and separated through a suspension hole 78 from each other.

The suspension plate 60 and load cell 81 are suspended through a rope 84, and further the load cell 81 is suspended on a hook 82 through a rope 83 while detecting the suspended

load, and this hook 82 is connected to a hoist not shown in the drawing. The hoist moves the entire suspension portion 64, in particular, moves vertically, and can move the upper end plate 61 and lower end plate 62 coupled together through the clamping tool 65 simultaneously.

On the suspension plate 60, an optical fiber 71 is wound through a take-up unit 73 corresponding to the vertical motion of the lower end plate 62. The optical fiber 71 penetrates the through-hole 76 of the suspension plate 60 and through-hole 75 of the upper end plate 61, and extends up to an imaging hole 94 of the lower end plate 62. At the leading end of the optical fiber 71, there is a lens 72 of the optical fiber 71, and the lens 72 is directed downward through the imaging hole 94. A weight 91 is provided at the leading end of the optical fiber 71, and the lens 72 of the optical fiber 71 is pressed, so that the image may be stably taken. The other end of the optical fiber 71 is connected to the imaging unit not shown through the take-up unit 73, and the image near the lens 72 is taken by the camera or the like.

The suspension plate 60 further has a detector 92 having a protrusion in the lower part. On the lower end plate 62 corresponding to this detector 92, a detecting protrusion 93 is formed, and this detecting protrusion 93 may abut against the lower protrusion of the detector 92 by way of

the through-hole 74 of the upper end plate 61. When this detecting protrusion 93 and the lower protrusion of the detector 92 abut against each other, the absorbing rod 11 is completely inserted into the control rod guide pipe 5 and measuring pipe 6. This abutting is detected by the detector 92, and the result of detection is issued to a detector not shown in the drawing.

Referring further to Fig. 9 to Fig. 11, the inserting operation of the absorbing rod 11 by the inserting apparatus is explained. The state shown in Fig. 7 is an initial state of insertion, and the upper end plate 61 is coupled to the suspension portion 64, and the suspender 63 is extended to the full extent. In this state, the relative configuration of the position of the lower end plate 62 and the position of the upper nozzle 2 is detected through the lens 72 of the optical fiber 71. This relative configuration is detected by image processing, and it is determined whether the relative position deviation is less than a specified value or not, for example, by pattern matching, and when less than the specified value, it is judged that the lower end plate 62 and upper nozzle 2 are matched in position, that is, the guide hole 77 of the lower end plate 62 and the control rod guide pipe 5 of the upper nozzle 2 are matched in relative configuration.

More specifically, as shown in Fig. 9A, a reference

image without deviation in relative position is obtained.
If the lower end plate 62 is deviated in position to upper
right direction in the drawing, as shown in Fig. 9B, the
passage 21 and control rod guide pipe 5 are positioned at
5 the lower right corner in the drawing, and the region of a
nozzle casing 100 at the upper end of the wall of the upper
nozzle 2 is wider. On the other hand, when the lower end
plate 62 is deviated to the lower right position in the
drawing, as shown in Fig. 9C, the passage 21 and the control
10 rod guide pipe 5 are positioned at the upper right corner
in the drawing, so that the region of the nozzle casing 100
is smaller. Therefore, the position deviation of the lower
end plate 62, that is, the position deviation of the absorbing
rod 11 can be detected by matching the patterns of the image
15 shown in Fig. 9B and Fig. 9C and the reference image shown
in Fig. 9A. This position deviation information is
transmitted to the operator handling the hoist, and this
operator handles so as to eliminate the position deviation
by manipulating the hoist on the basis of this information
20 of position deviation.

When correction of position deviation is over, the
operator lowers the hoist, and also lowers the upper end plate
61, thereby lowering the absorbing rod 11, so that the
absorbing rod 11 is inserted into the control rod guide pipe
25 5 and measuring pipe 6 through the guide hole 77. In this

case, the lower end plate 62 is mounted on the inner upper side of the upper nozzle 2 at a position free from deviation of position.

When insertion of absorbing rod 11 into the control
5 rod guide pipe 5 and others is over, the protrusion of the detector 92 and the detecting protrusion 93 abut against each other, and the result of detection is issued to the detector not shown. On the basis of this result of detection, the operator manipulates to extend the air cylinder 67, and
10 eliminate hooking of the claw 69. Then, the operator lifts the hoist, and separates the upper end plate 61 from the suspension portion 64. Whether separated actually or not can be checked by the load cell 81.

As a result, as shown in Fig. 11, the absorbing rod
15 11 is inserted into the control rod guide pipe 5 and others, and the upper end plate 61 holding the absorbing rod 11 and the lower end plate 62 guiding the absorbing rod 11 settle within the upper nozzle 2, thereby not exceeding the shape of the upper nozzle 2. Therefore, these spent fuel
20 assemblies are in the same shape as the spent fuel assemblies in which the absorbing rods 11 are not inserted.

Incidentally, the abutting of the detector 92 and detecting protrusion 93 can be detected by magnetic or electric means. Alternatively, using an optical detector
25 as the detector 92, the distance can be measured by optical

means, and it may be judged that the insertion is complete when the distance becomes shorter than a specified value.

By forming the upper end plate 61 and lower end plate 62 by a material having a neutron absorbing capability such as B-Al material, same as the absorbing rods 11, it is possible to reduce the thickness of the resin in the lid direction in the case of a wet cask for storing in a state immersed in water.

According to the third embodiment, by detecting the relative position deviation of the absorbing rods and control rod guide pipes 5 and others by using the inserting apparatus, the positions are corrected approximately, and the relative positions of the absorbing rods and control rod guide pipes 5 and others are matched, and further by using the tape shape of the lading end of the absorbing rods, the absorbing rods can be securely and easily inserted into the control rod guide pipes 5. Also by detecting the completion of insertion, safety in insertion can be assured.

A forth embodiment is explained below. In the second embodiment, it is intended to store the spent fuel assemblies in the first embodiment into the cask. However, the forth embodiment specifies the method of storing spent fuel assemblies in the storage pool.

Fig. 12 is a schematic diagram showing configuration when storing fuel assemblies 101 as spent fuel assemblies

in a pool 102. As shown in Fig. 12, the fuel assembly 101 in which the absorbing rod 11 shown in the first embodiment is inserted is stored in a rack cell in a rack 104 arranged in the pool 102 filled with water 103. Herein, the fuels
5 are stored in the pool 103 while mutually keeping a safe spacing distance d for assuring the subcriticality of the fuels, but since the absorbing rods 11 having neutron absorbing capability are inserted in the fuel assemblies 101, the spacing distance d may shorter than the spacing distance
10 d when the absorbing rods 11 are not inserted.

Therefore, according to the forth embodiment, the fuel assemblies 101 can be arranged at high density, and the number of fuel assemblies 101 to be stored is increased, or the size of the pool 102 can be reduced.

15 In the first to forth embodiments, spent fuel assemblies for PWR of 14×14 are shown as examples, but not limited to this example, it is evident to be applicable similarly in other spent fuel assemblies for PWR, such as 17×17 or 15×15 arrangement as shown in Fig. 16.

20 As described herein, according to the absorbing rod of one aspect of the present invention, using the absorbing rod having a nearly same shape as the shape of a columnar control rod for PWR used in reactivity control of core in a reactor, and having a neutron shielding capability, this
25 absorbing rod is inserted into the control rod guide pipe

of spent fuel assemblies or the control rod guide pipe including the measuring pipe, and therefore the neutrons can be shielded effectively in the spent fuel assemblies.

According to the absorbing rod of another aspect of the present invention, it is made of aluminum composite material or aluminum alloy formed by adding powder of boron or boron compound having a neutron absorbing performance to aluminum or aluminum alloy powder. Since the absorbing rod is made of such material, it does not get damaged by minor shocks during transportation. As a consequence, it is possible to maintain the neutron absorbing ability even during transport. For example, it is much safer and therefore preferable than an absorbing rod having a structure in which boron powder is filled inside a pipe.

According to the cask of still another aspect of the present invention, by inserting the absorbing rod group having a nearly same shape as the shape of a columnar control rod for PWR used in reactivity control of core in a reactor, and having a neutron shielding capability into the control rod guide pipe of spent fuel assemblies or the control rod guide pipe group including the measuring pipe, and putting the spent fuel assemblies into the cask, the number of neutrons radiated outside from the spent fuel assemblies can be decreased, so that the spacing distance among spent fuel assemblies can be shortened, and therefore it is effective

to increase the number of spent fuel assemblies to be accommodated in the cavity of the shell main body having the neutron shield on the outer circumference and shielding the gamma-rays, or in the same number of containers as in the prior art, the cavity shape of the shell main body may be reduced or the thickness of the neutron shield on the outer circumference can be reduced, thereby reducing the overall size of the cask.

According to the cask of still another aspect of the present invention, when the control rods for PWR are recovered, the absorbing rod is used as the control rod for PWR, and the control rods are kept inserted in the spent fuel assemblies, and therefore the recovered control rods can be utilized effectively.

Further, since the equivalent thickness corresponding to the neutron absorbing capability or neutron moderating capability of the plates or square pipes for composing the basket for holding the spent fuel assemblies for PWR in the cask, of the portion corresponding to the sectional area of the absorbing rod inserted in the spent fuel assemblies for PWR, is reduced for the portion of the equivalent sectional area corresponding to the neutron absorbing capability or neutron moderating capability of the absorbing rod inserted in the spent fuel assemblies for PWR, thereby forming in a basket shape, the water zones provided in the basket for PWR

high in uranium enrichment factor, in particular, can be further reduced in height, or the basket may be formed even by eliminating the water zones, and therefore the shape of the basket for PWR high in uranium enrichment factor may be formed same as a simple shape of basket for BWR, and the efficiency of basket production can be enhanced.

Further, since the absorbing rod and basket are composed of an Al composite material or an Al alloy formed by adding powder of B or B compound having a neutron absorbing performance to Al or Al alloy powder, neutrons are absorbed in side and around the spent fuel assemblies, and therefore, aside from efficient neutron absorption in and around the spent fuel assemblies, since the absorbing rods are inserted in the fuel assemblies, the required performance in strength is lower, so that the weight reduction can be promoted.

According to the inserting apparatus of still another aspect of the present invention, the relative positions of the through-hole group of the guide member and the control rod guide pipe group of the spent fuel assemblies are adjusted on the basis of the result of detection by the position detection unit, and the absorbing rod block is lowered so that the through-hole group acts as a guide hole, and the long absorbing rod group can be inserted securely into the control rod guide pipe or the control rod guide pipe including the measuring pipe, and therefore the absorbing rods can be

inserted into the control rod guide pipes and others easily and securely, so that the efficiency of inserting job can be enhanced.

Further, the distance detection unit detects when the distance between the suspender and the fixing member comes within a specific distance, and it is known that the absorbing rod group is securely inserted into the control rod guide pipe, so that dropping of absorbing rod group in the midst of insertion is prevented, and therefore the efficiency of inserting job can be enhanced, and the end of series of inserting job can be detected securely, so that the inserting job can be done safely.

According to the conveying and storing method of still another aspect of the present invention, at the inserting step, by inserting an absorbing rod group having a nearly same shape as the shape of a columnar control rod for PWR used in reactivity control of core in a reactor, and having a neutron shielding capability, into a control rod guide pipe group including a measuring pipe of spent fuel assemblies for PWR, and at the conveying and storing step, by conveying and storing the spent fuel assemblies for PWR in a state in which the absorbing rod group is inserted, the spacing distance between spent fuel assemblies can be shortened, and therefore the density of the spent fuel assemblies for PWR when conveying or storing the spent fuel assemblies for PWR

can be enhanced, and efficient conveying or storing may be possible.

Further, when the control rods for PWR are recovered, the absorbing rod is used as the control rod for PWR, and
5 the control rods are kept inserted in the spent fuel assemblies, and therefore the recovered control rods can be utilized effectively.

Although the invention has been described with respect to a specific embodiment for a complete and clear disclosure,
10 the appended claims are not to be thus limited but are to be construed as embodying all modifications and alternative constructions that may occur to one skilled in the art which fairly fall within the basic teaching herein set forth.

WHAT IS CLAIMED IS:

1. An absorbing rod having a nearly the same shape as the shape of a columnar control rod for PWR used in reactivity control of core in a reactor, and having a neutron shielding
5 capability.

2. The absorbing rod according to claim 1, further comprising a fixing member for fixing said absorbing rod at a position corresponding to a control rod guide pipe of spent
10 fuel assemblies for PWR or a sectional position of the control rod guide pipe including a measuring pipe.

3. The absorbing rod according to claim 1, wherein said absorbing rod is composed of aluminum composite material or
15 aluminum alloy formed by adding powder of boron or boron compound having a neutron absorbing performance to aluminum or aluminum alloy powder.

4. An absorbing rod which is to be inserted into a control
20 rod guide pipe of spent fuel assemblies or a measuring pipe, wherein

said absorbing rod is composed of aluminum composite material or aluminum alloy formed by adding powder of boron or boron compound having a neutron absorbing performance to
25 aluminum or aluminum alloy powder, and

said absorbing rod is used, when transporting the spent fuel assemblies by storing into a cask, by inserting said absorbing rod into said control rod guide pipe or said measuring pipe.

5

5. A cask comprising:

an absorbing rod group, to be inserted into a control rod guide pipe group including a control rod guide pipe or a measuring pipe of spent fuel assemblies for PWR, having a nearly same shape as the shape of a columnar control rod for PWR used in reactivity control of core in a reactor, and having a neutron shielding capability;

spent fuel assemblies for PWR having a control rod guide pipe in which said absorbing rod group is inserted or a control rod guide pipe group including a measuring pipe;

a basket forming a lattice cell for accommodating the spent fuel assemblies for PWR in which said absorbing rod group is inserted;

a shell main body accommodated in said basket, and having a neutron shield on the outer circumference, for shielding gamma-rays; and

a detachable lid provided in an opening of said shell main body for putting the spent fuel assemblies for PWR in which said absorbing rod group is inserted, in and out of said lattice cell.

6. The cask according to claim 5, wherein said absorbing rod is a control rod for PWR.

7. The cask according to claim 5, wherein the equivalent
5 thickness corresponding to the neutron absorbing capability
or neutron moderating capability of the plates or square
pipes for composing the basket for holding the spent fuel
assemblies for PWR in the cask, of the portion corresponding
to the sectional area of the absorbing rod inserted in the
10 spent fuel assemblies for PWR, is reduced for the portion
of the equivalent sectional area corresponding to the
neutron absorbing capability or neutron moderating
capability of the absorbing rod inserted in the spent fuel
assemblies for PWR.

15

8. The cask according to claim 5, wherein said absorbing
rod is composed of aluminum composite material or an aluminum
alloy formed by adding powder of boron or born compound having
a neutron absorbing performance to aluminum or aluminum
20 alloy powder.

9. An inserting apparatus comprising:
an absorbing bar element having a nearly same shape
as the shape of a columnar control rod for PWR used in
25 reactivity control of core, at a position corresponding to

a control rod guide pipe of spent fuel assemblies for PWR
or a sectional position of the control rod guide pipe
including a measuring pipe, capable of inserting the fixing
member into an upper nozzle through the upper nozzle opening
5 of the spent fuel assemblies for PWR, by fixing the absorbing
rod group having a neutron shielding capability by means of
the fixing member;

a guide member which can be inserted into the upper
nozzle through the upper nozzle opening of the spent fuel
10 assemblies for PWR, having a through-hole for inserting said
absorbing rod group into the control rod guide pipe or a
position corresponding to the sectional position of the
control rod guide pipe including the measuring pipe, so as
to be suspended on said fixing member in a state of the leading
15 end of the absorbing rod group being inserted into the
through-hole when standing still;

a suspender detachably suspending the fixing member
of said absorbing rod block and moving the absorbing rod
block vertically when mounting, and

20 a position detection unit provided in said guide member
which detects the relative configuration of the location of
the through-hole group provided in the guide member and the
location of the corresponding control rod guide pipe group
of the spent fuel assemblies for PWR,

25 wherein the relative positions of the through-hole

group and control rod guide pipe group are adjusted on the basis of the result of detection by said position detection unit, and the absorbing rod group is lowered to insert the absorbing rod group into the control rod guide pipe or the control rod guide pipe including the measuring pipe.

10. The inserting apparatus according to claim 9, further comprising a distance detection unit which detects whether the distance between the suspender and the fixing member is within a specific range or not.

11. A conveying and storing method of spent fuel assemblies comprising:

a step of inserting an absorbing rod group having a nearly same shape as the shape of a columnar control rod for PWR used in reactivity control of core in a reactor, and having a neutron shielding capability, into a control rod guide pipe group including a measuring pipe of spent fuel assemblies for PWR; and

a step of conveying and storing the spent fuel assemblies for PWR in a state in which said absorbing rod group is inserted.

12. The conveying and storing method of spent fuel assemblies according to claim 11, wherein said absorbing rod is a control rod for PWR.

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ABSTRACT OF THE DISCLOSURE

The absorbing rods have a nearly the same shape as the shape of columnar control rods for PWR used in reactivity control of core in a reactor. The absorbing rods can shield
5 neutrons, and are inserted in control rod guide pipes and measuring pipes in fuel assemblies.

CONFIDENTIAL

Title : "SPENT-FUEL STORAGE CONTAINER BY MITSUBISHI HEAVY INDUSTRIES, LTD.", Nuclear Viewpoints Vol.44/No. 4

Author : Kunio MARUOKA of MITSUBISHI HEAVY INDUSTRIES, LTD.

Publication date : April 1998

5 Publisher : Nikkan Kogyo Syuppan Production

MSF-V developed by MITSUBISHI HEAVY INDUSTRIES, LTD.

10 This paper deals with the spent-fuel storage container, MSF-V.

(1) Fig. 1 shows a bird's-eye view of storage container. Table 1 shows the design specification.

15 (2) The storage container is designed to store 21 highly-burnup spent-fuel rods for PWR or 52 rods for BWR. Further increase in number of storage rods is possible as the number of cooling years is increased.

(3) The storage container consists of the body, lid, basket, etc. in the structure described below.

20 I. Body

The body comprises the multi-layer-drum, upper flange, bottom plate, etc. The drum is composed of the structural materials, namely, the inner cylinder, intermediate drum and outer cylinder, as well as the lead material that shields gamma
25 rays and high-polymer material with polonium that shields

neutron. The sealing boundary of the storage container is formed by the inner cylinder, bottom plate, and primary lid that is described later.

The structural materials are made from stainless steel or carbon steel. The outer cylindrical surface has no heat-radiation fins. Additionally, internal heat-conduction fins (that provide direct heat transmission between the intermediate drum and outer cylinder) are buried into the neutron shield material for improved heat-conductivity.

II. Lid

The lid comprises the primary lid and the secondary lid, which are bolted to the body, respectively. Each of those lids is provided with the metallic O-ring and elastomer O-ring as a gasket set. When a storage container is used for storing spent fuel, the space between the primary lid and the secondary lid is pressurized with an inert gas, such as helium or the like, and gas pressure is always monitored with a pressure gauge attached to the lid.

III. Basket

The basket for PWR is composed of the cells (that receive spent-fuel rods), made of stainless steel holding polonium component, and the copper plates that radiate heat. For neutron absorption material, aluminum alloy containing

polonium may also be used.

Fig. 1 Structure of MSF-V type storage container

- 5 1. Lower buffer
2. Lower trunnion
3. Heat-radiation fins
4. Outer cylinder
5. Neutron shield
- 10 6. Intermediate drum
7. γ -rays shield
8. Inner cylinder
9. Fuel assembly (PWR)
10. Upper trunnion
- 15 11. Basket
12. Primary lid
13. Secondary lid
14. Upper buffer

FIG.1

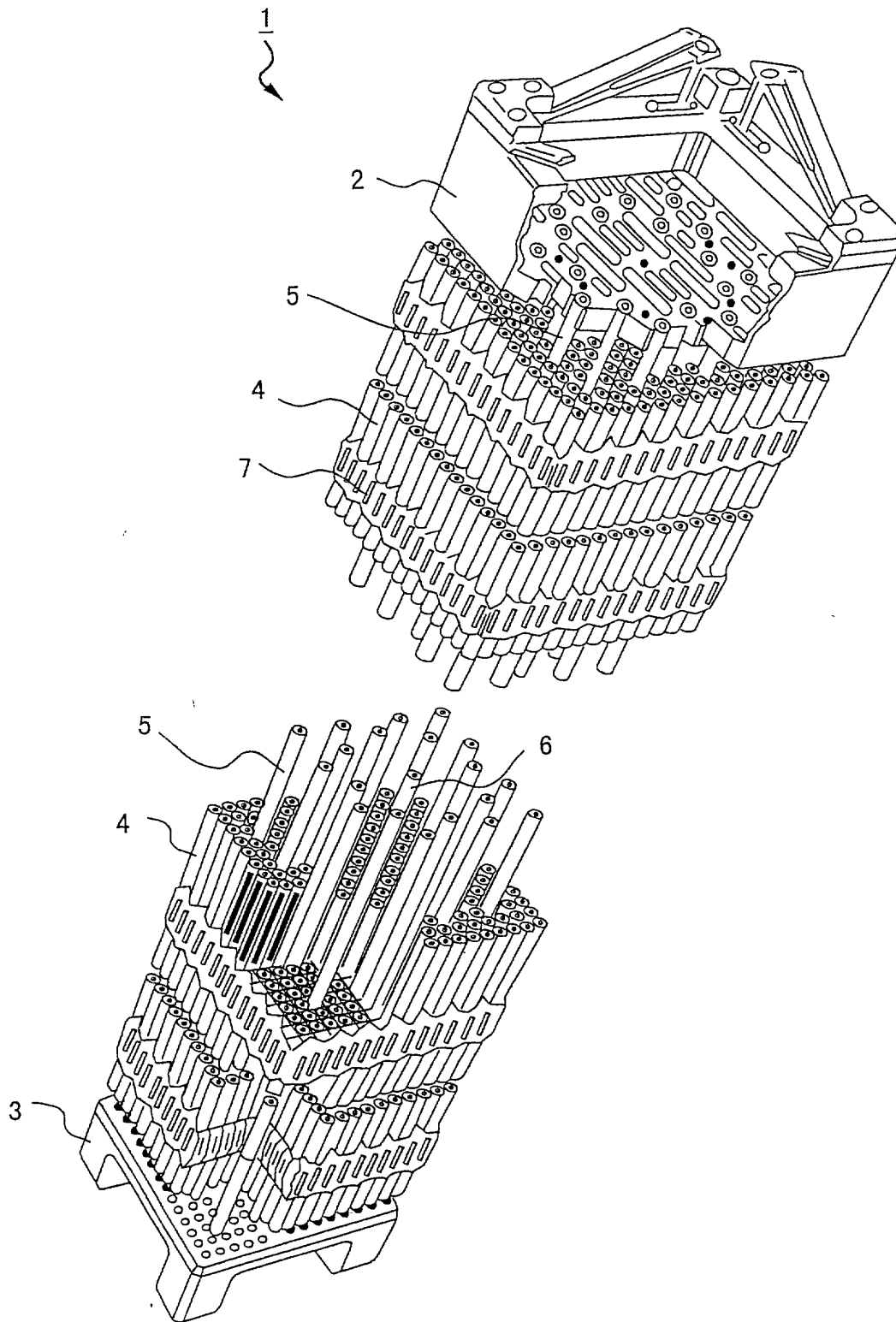


FIG.2

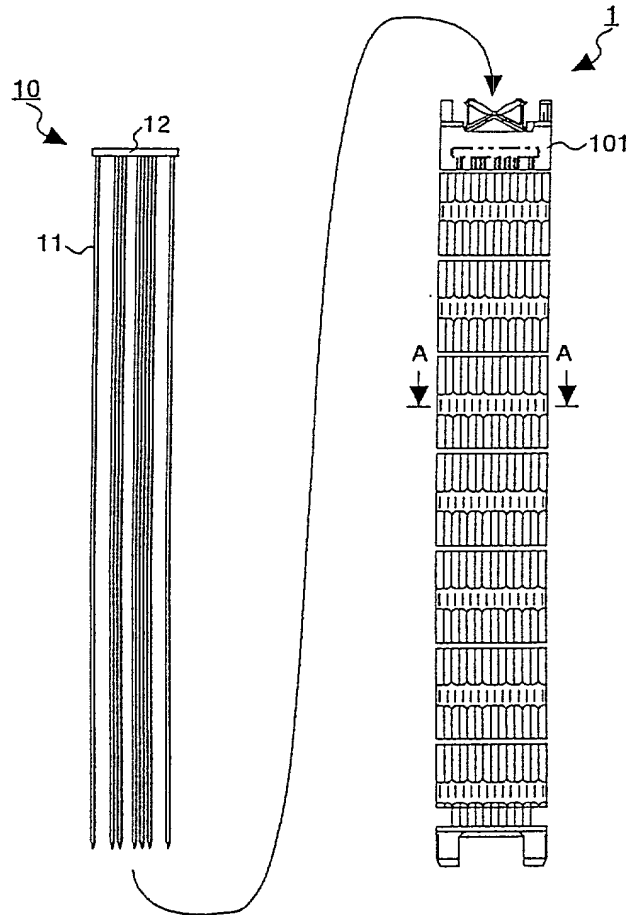


FIG.3

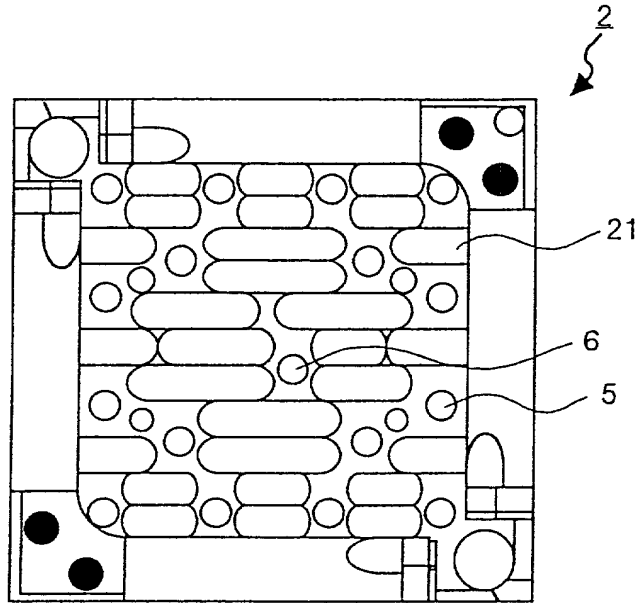
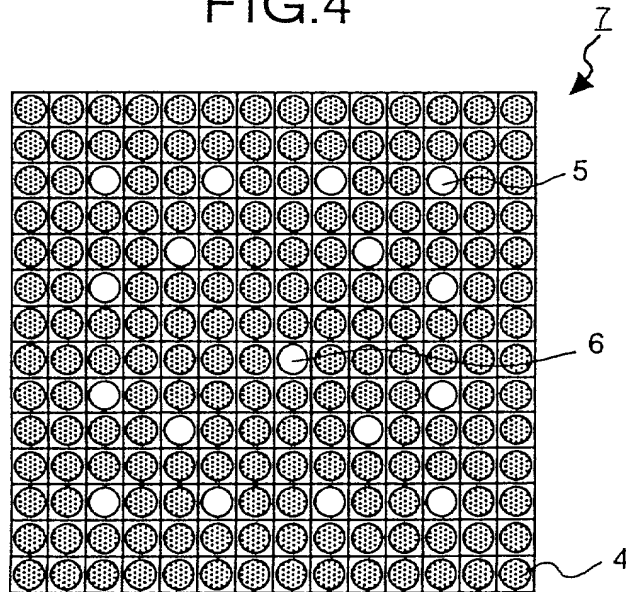


FIG.4



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FIG. 5

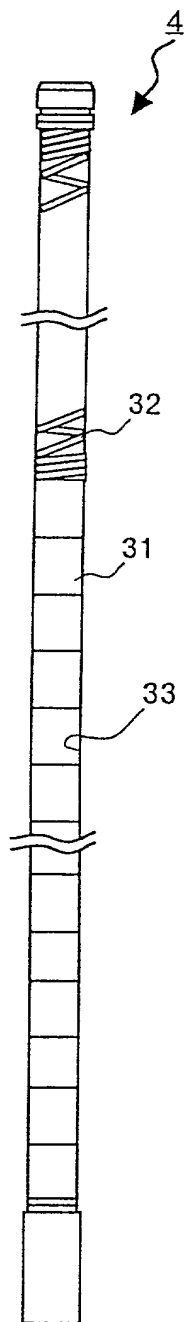


FIG.6

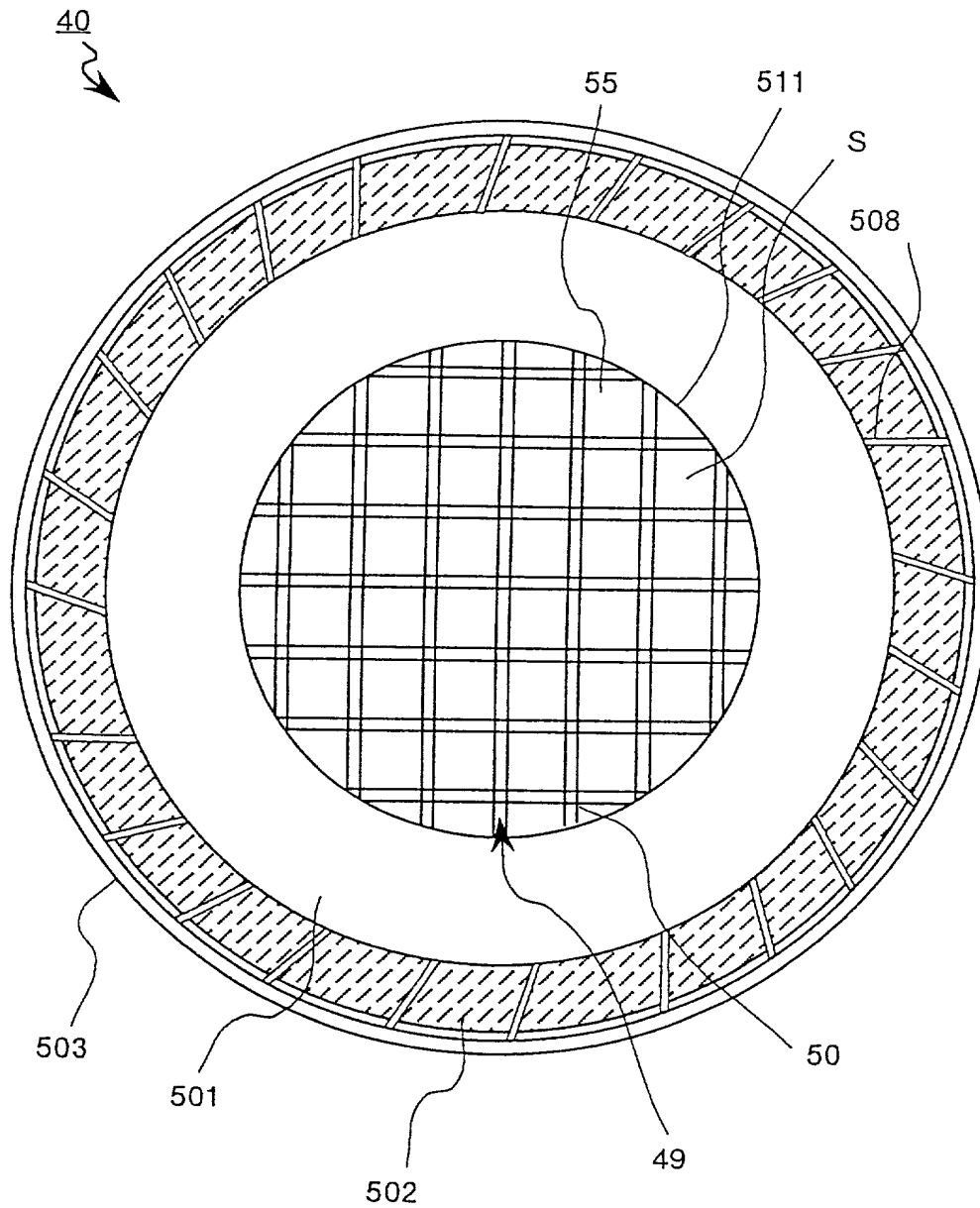


FIG.7

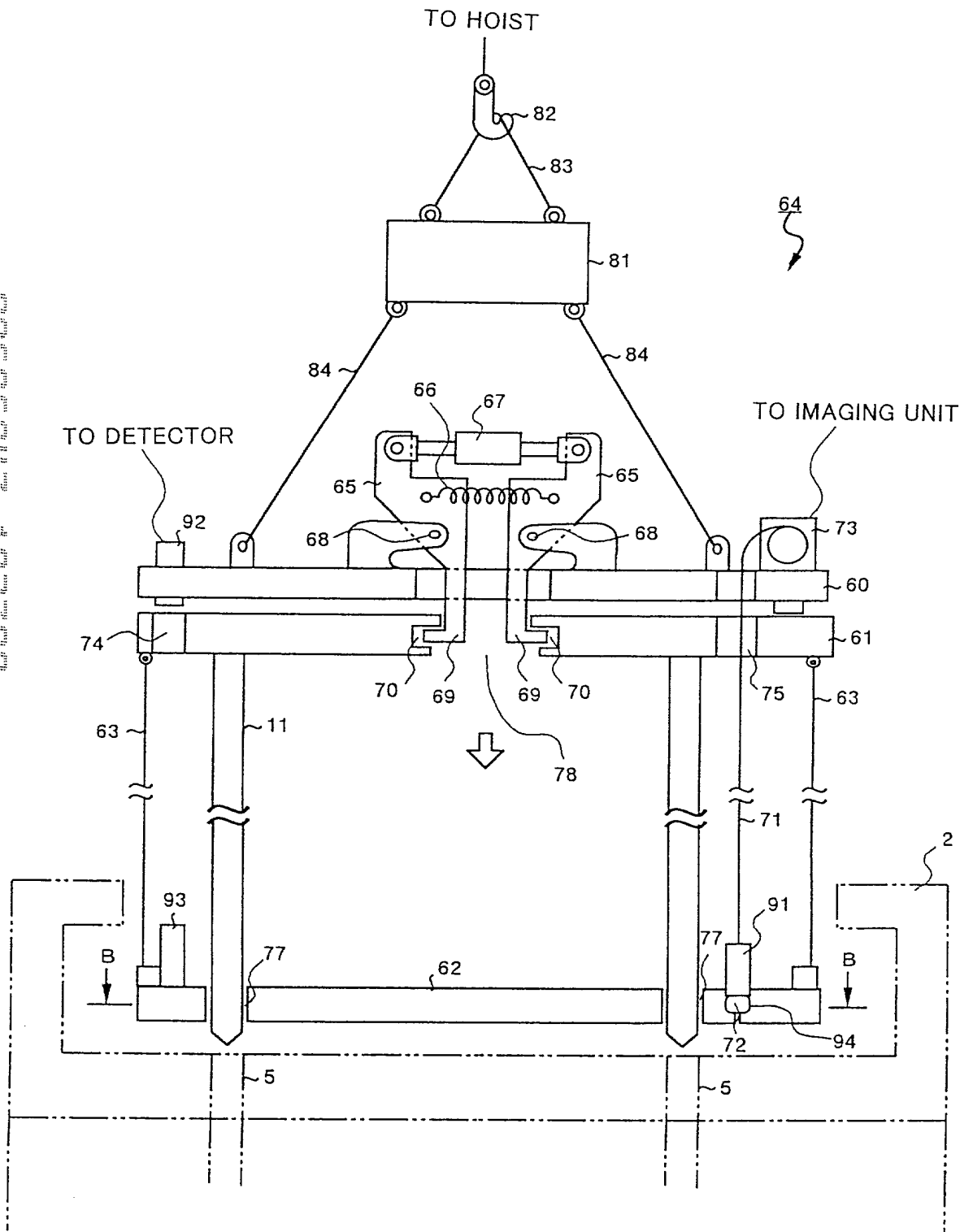


FIG.8

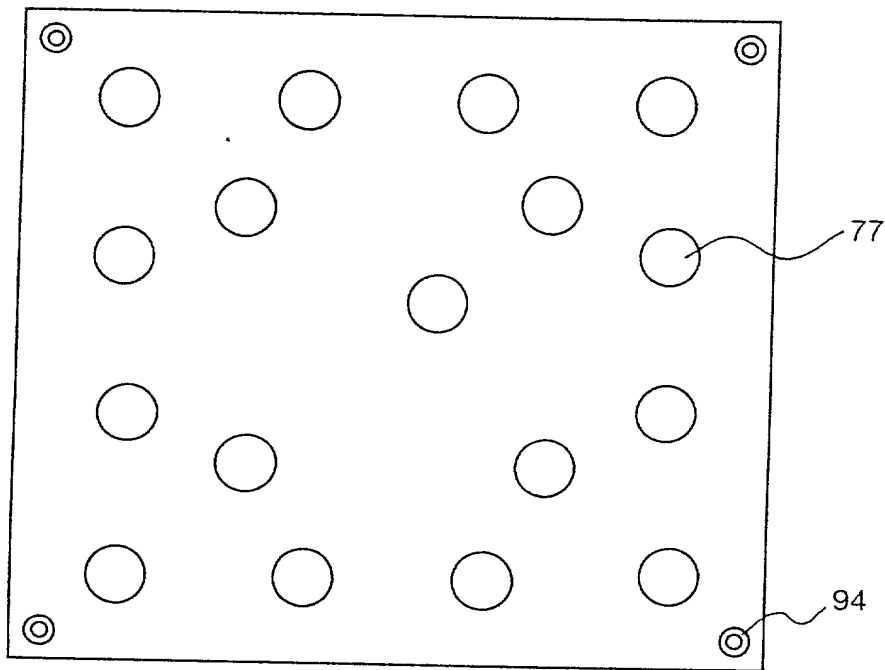


FIG.9A

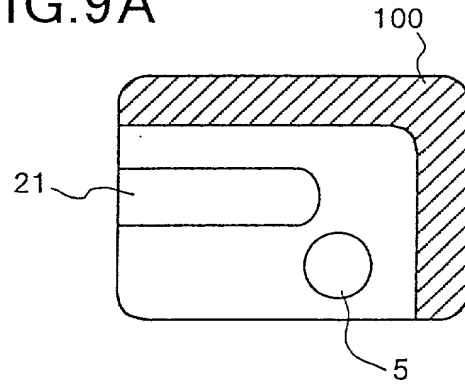


FIG.9B

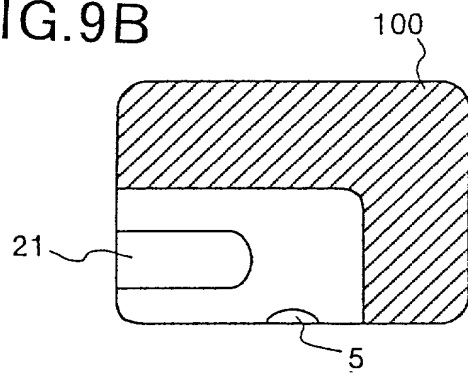


FIG.9C

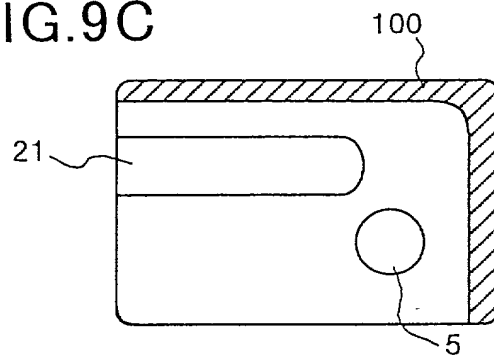


FIG.10

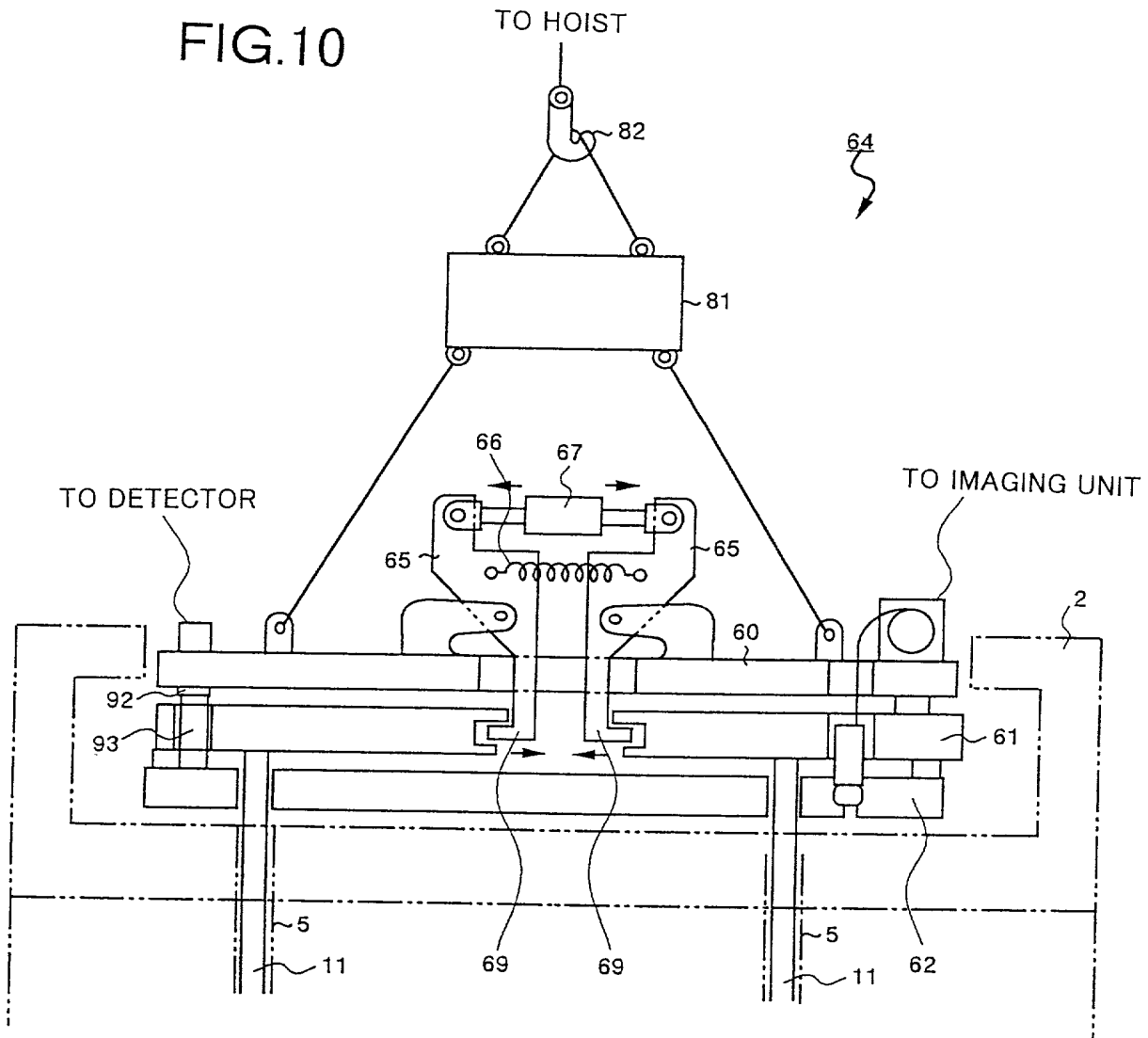
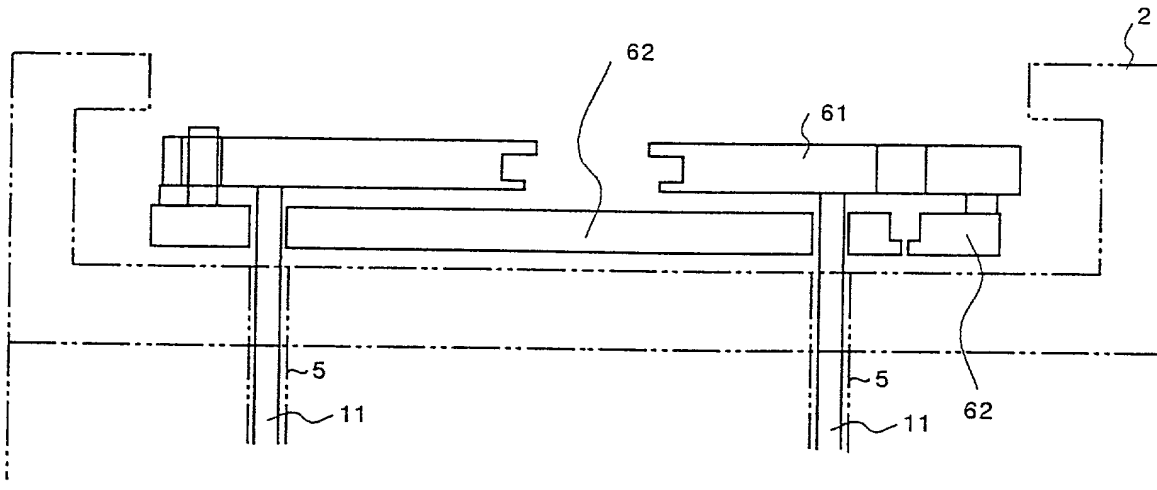
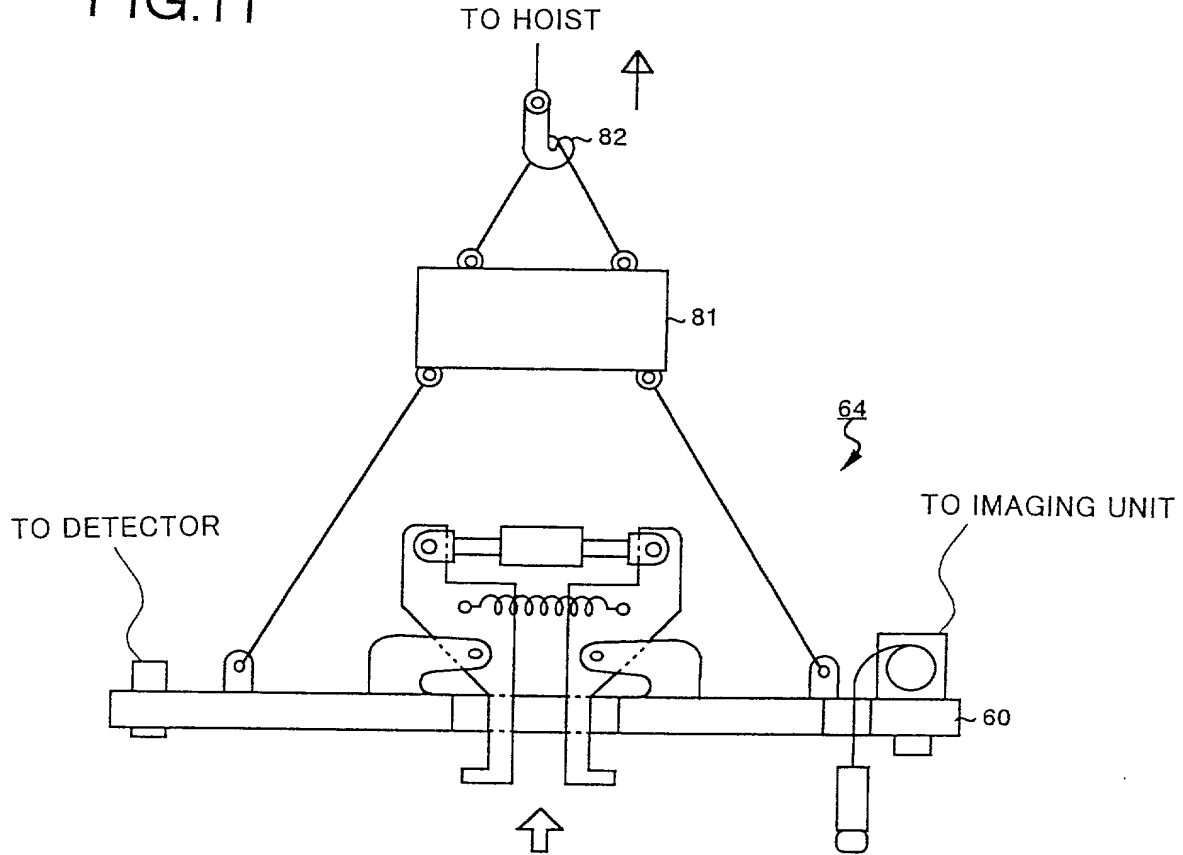


FIG. 11



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FIG.12

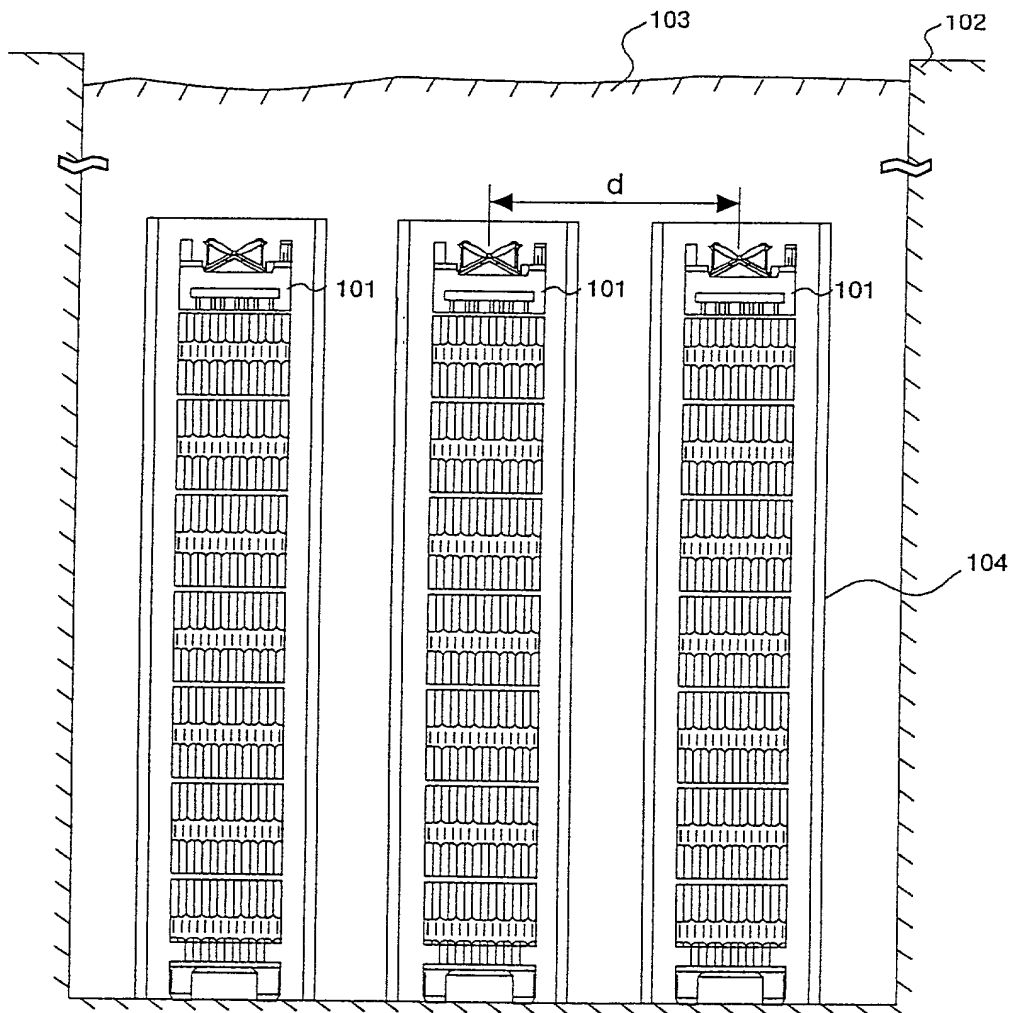




FIG.13

FIG.14

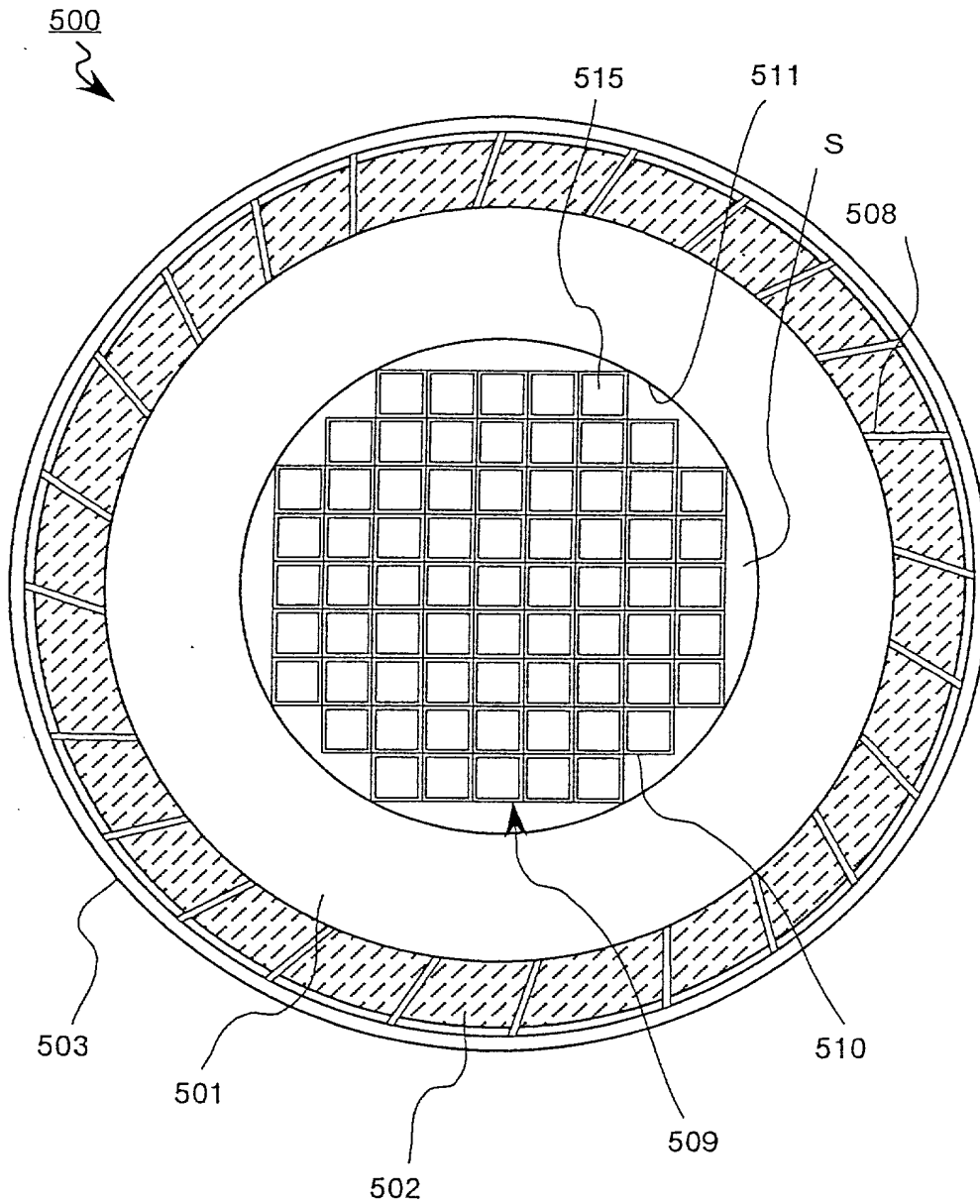


FIG. 15

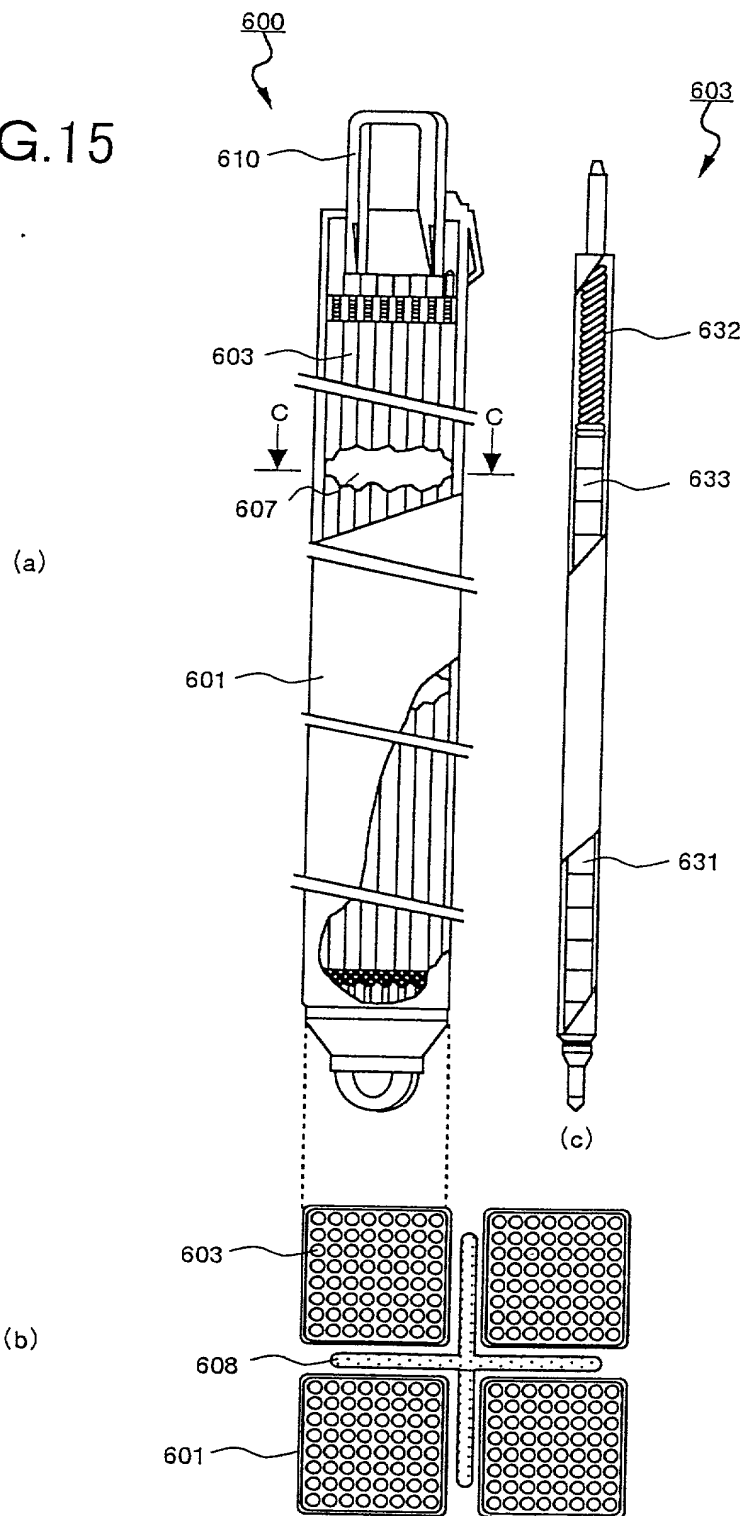


FIG.16

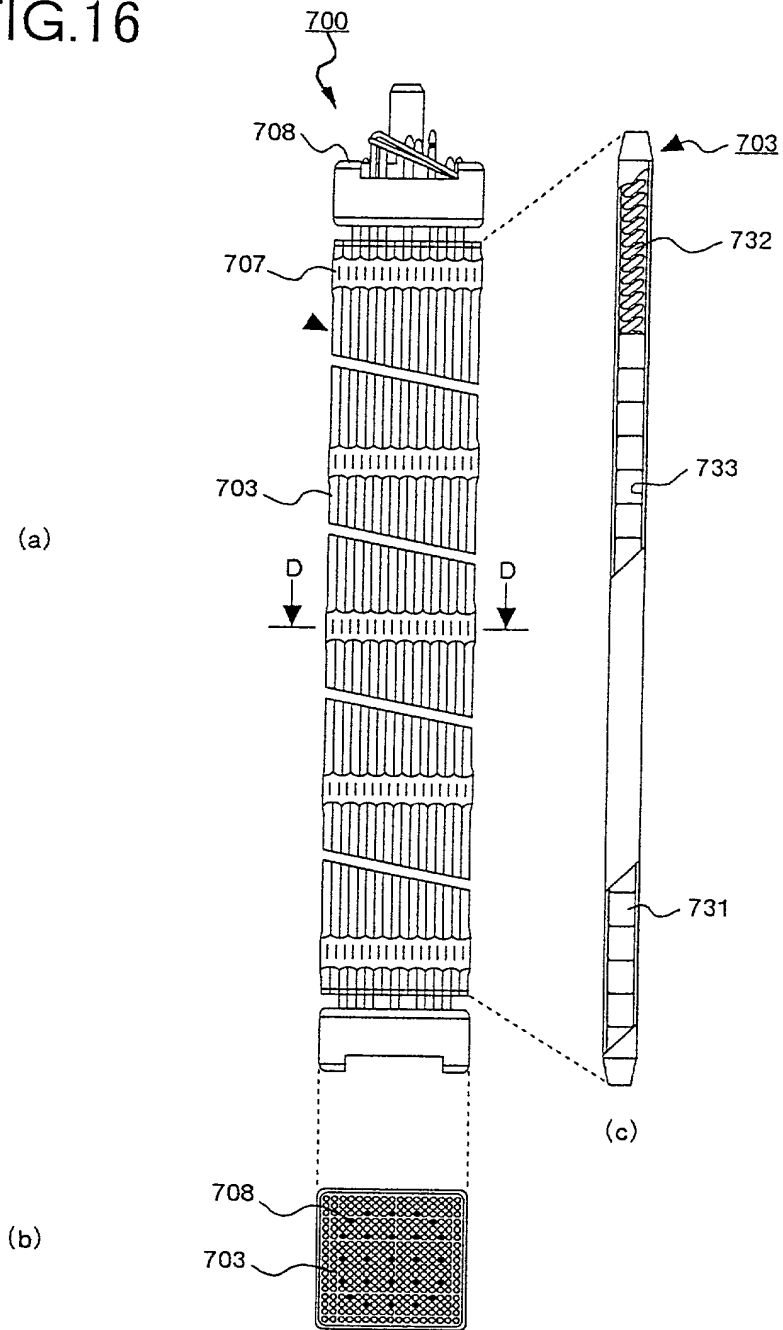
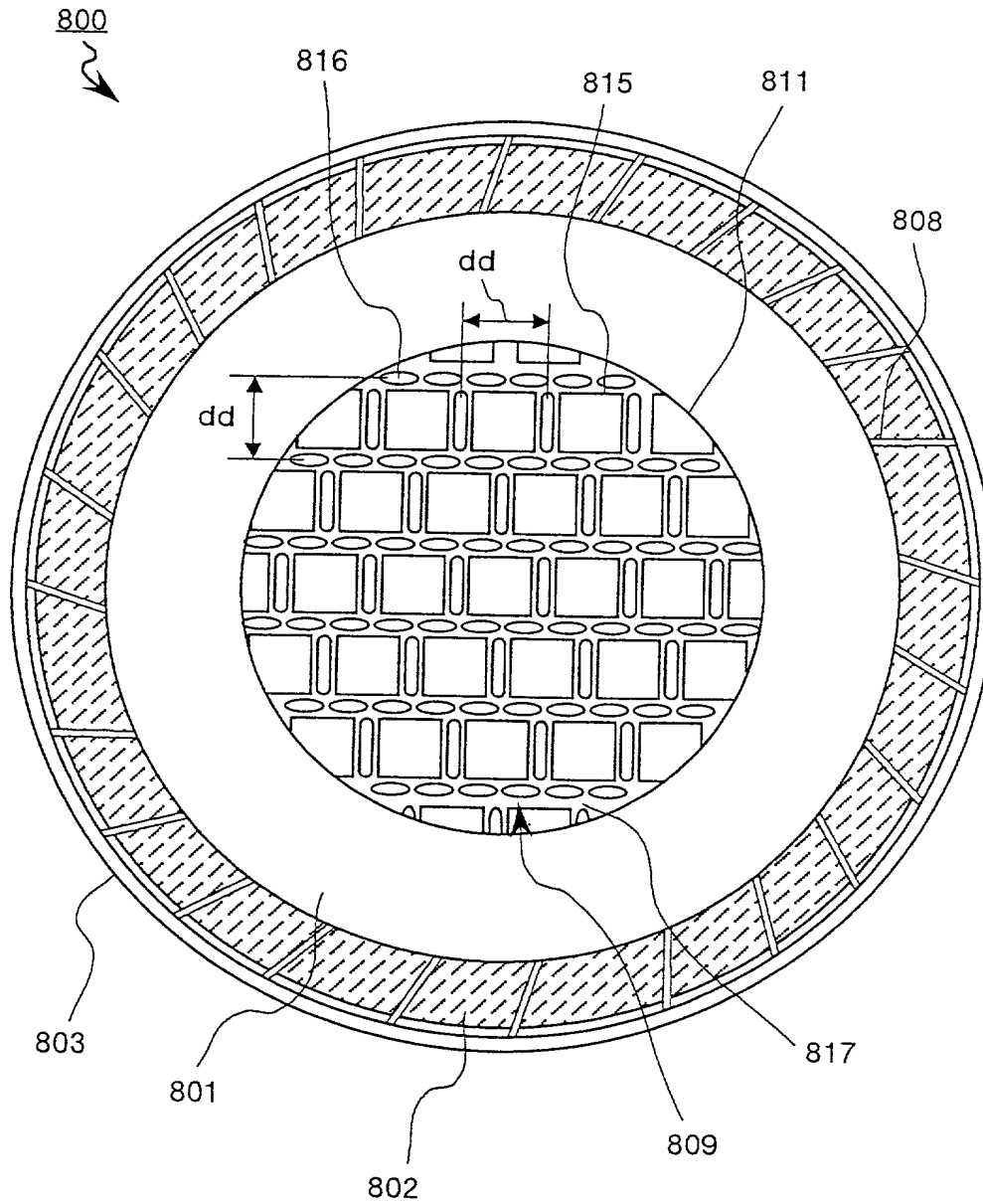


FIG.17



Declaration and Power of Attorney For Patent Application

特許出願宣言書及び委任状

Japanese Language Declaration

日本語宣言書

下記の氏名の発明者として、私は以下の通り宣言します。

As a below named inventor, I hereby declare that:

私の住所、私書箱、国籍は下記の私の氏名の後に記載された通りです。

My residence, post office address and citizenship are as stated next to my name.

下記の名称の発明に関して請求範囲に記載され、特許出願している発明内容について、私が最初かつ唯一の発明者（下記の氏名が一つの場合）もしくは最初かつ共同発明者（下記の名称が複数の場合）であると信じています。

I believe I am the original, first and sole inventor (if only one name is listed below) or an original, first and joint inventor (if plural names are listed below) of the subject matter which is claimed and for which a patent is sought on the invention entitled.

AN ABSORBING ROD, AN APPARATUS FOR
INSERTING THE ABSORBING ROD, A CASK, AND
A METHOD OF STORING SPENT FUEL
ASSEMBLIES

上記発明の明細書は、

- ☐ 本書に添付されています。
- ☐ ____月____日に提出され、米国出願番号または特許協定条約国際出願番号を____とし、
(該当する場合) ____に訂正されました。

the specification of which

☒ is attached hereto.

☐ was filed on _____
as United States Application Number or
PCT International Application Number
_____ and was amended on
_____ (if applicable).

私は、特許請求範囲を含む上記訂正後の明細書を検討し、内容を理解していることをここに表明します。

I hereby state that I have reviewed and understand the contents of the above identified specification, including the claims, as amended by any amendment referred to above.

私は、連邦規則法典第37編第1条56項に定義されるとおり、特許資格の有無について重要な情報を開示する義務があることを認めます。

I acknowledge the duty to disclose information which is material to patentability as defined in Title 37, Code of Federal Regulations, Section 1.56.

Japanese Language Declaration
(日本語宣言書)

私は、米国法典第35編119条 (a) - (d) 項又は365条 (b) 項に基づき下記の、米国以外の国の少なくとも一カ国を指定している特許協力条約365 (a) 項に基づく国際出願、又は外国での特許出願もしくは発明者証の出願についての外国優先権をここに主張するとともに、優先権を主張している、本出願の前に出願された特許または発明者証の外国出願を以下に、枠内をマークすることで、示しています。

Prior Foreign Application(s)
外国での先行出願

11-311713

Japan

(Number)
(番号)

(Country)
(国名)

(Number)
(番号)

(Country)
(国名)

私は、第35編米国法典119条 (e) 項に基づいて下記の米国特許出願規定に記載された権利をここに主張いたします。

(Application No.)
(出願番号)

(Filing Date)
(出願日)

私は、下記の米国法典第35編120条に基づいて下記の米国特許出願に記載された権利、又は米国を指定している特許協力条約365条 (c) に基づく権利をここに主張します。また、本出願の各請求範囲の内容が米国法典第35編112条第1項又は特許協力条約で規定された方法で先行する米国特許出願に開示されていない限り、その先行米国出願書提出日以降で本出願書の日本国内または特許協力条約国際提出日までの期間中に入手された、連邦規則法典第37編1条56項で定義された特許資格の有無に関する重要な情報について開示義務があることを認識しています。

(Application No.)
(出願番号)

(Filing Date)
(出願日)

(Application No.)
(出願番号)

(Filing Date)
(出願日)

私は、私自信の知識に基づいて本宣言書中で私が行なう表明が真実であり、かつ私の入手した情報と私の信じることに基づく表明が全て真実であると信じていること、さらに故意になされた虚偽の表明及びそれと同等の行為は米国法典第18編第1001条に基づき、罰金または拘禁、もしくはその両方により処罰されること、そしてそのような故意による虚偽の声明を行えば、出願した、又は既に許可された特許の有効性が失われることを認識し、よってここに上記のごとく宣誓を致します。

I hereby claim foreign priority under Title 35, United States Code, Section 119 (a)-(d) or 365(b) of any foreign application(s) for patent or inventor's certificate, or Section 365(a) of any PCT International application which designated at least one country other than the United States, listed below and have also identified below, by checking the box, any foreign application for patent or inventor's certificate, or PCT International application having a filing date before that of the application on which priority is claimed.

Priority Claimed
優先権主張

1 / November / 1999

(Day/Month/Year Filed)
(出願年月日)

☒

Yes
はい

☐

No
いいえ

☐

Yes
はい

☐

No
いいえ

I hereby claim the benefit under Title 35, United States Code, Section 119(e) of any United States provisional application(s) listed below.

(Application No.)
(出願番号)

(Filing Date)
(出願日)

I hereby claim the benefit under Title 35, United States Code, Section 120 of any United States application(s), or Section 365(c) of any PCT International application designating the United States, listed below and, insofar as the subject matter of each of the claims of this application is not disclosed in the prior United States or PCT International application in the manner provided by the first paragraph of Title 35, United States Code Section 112, I acknowledge the duty to disclose information which is material to patentability as defined in Title 37, Code of Federal Regulations, Section 1.56 which became available between the filing date of the prior application and the national or PCT International filing date of application.

(Status: Patented, Pending, Abandoned)
(現況: 特許許可済、係属中、放棄済)

(Status: Patented, Pending, Abandoned)
(現況: 特許許可済、係属中、放棄済)

I hereby declare that all statements made herein of my own knowledge are true and that all statements made on information and belief are believed to be true; and further that these statements were made with the knowledge that willful false statements and the like so made are punishable by fine or imprisonment, or both, under Section 1001 of Title 18 of the United States Code and that such willful false statements may jeopardize the validity of the application or any patent issued thereon.

Japanese Language Declaration
(日本語宣言書)

委任状：私は下記の発明者として、本出願に関する一切の手続きを米特許商標局に対して遂行する弁理士または代理人として、下記の者を指名いたします。
(弁理士、または代理人の指名及び登録番号を明記のこと)

POWER OF ATTORNEY: As a named inventor, I hereby appoint the following attorney(s) and/or agent(s) to prosecute this application and transact all business in the Patent and Trademark Office connected therewith: (list name and registration number)

Norman F. Oblon, Reg. No. 24,618; Marvin J. Spivak, Reg. No. 24,913; C. Irvin McClelland, Reg. No. 21,124; Gregory J. Maier, Reg. No. 25,599; Arthur I. Neustadt, Reg. No. 24,854; Richard D. Kelly, Reg. No. 27,757; James D. Hamilton, Reg. No. 28,421; Eckhard H. Kuesters, Reg. No. 28,870; Robert T. Pous, Reg. No. 29,099; Charles L. Gholz, Reg. No. 26,395; William E. Beaumont, Reg. No. 30,996; Jean-Paul Lavalleye, Reg. No. 31,451; Stephen G. Baxter, Reg. No. 32,884; Richard L. Treanor, Reg. No. 36,379; Steven P. Weihrouch, Reg. No. 32,829; John T. Goolkasian, Reg. No. 26,142; Richard L. Chinn, Reg. No. 34,305; Steven E. Lipman, Reg. No. 30,011; Carl E. Schlier, Reg. No. 34,426; James J. Kulbaski, Reg. No. 34,648; Richard A. Neifeld, Reg. No. 35,299; J. Derek Mason, Reg. No. 35,270; Surinder Sachar, Reg. No. 34,423; Christina M. Gadiano, Reg. No. 37,628; Jeffrey B. McIntyre, Reg. No. 36,867; William T. Enos, Reg. No. 33,128; Michael E. McCabe, Jr., Reg. No. 37,182; Bradley D. Lytle, Reg. No. 40,073; and Michael R. Casey, Reg. No. 40,294, with full powers of substitution and revocation.

書類送付先

Send Correspondence to:

OBLON, SPIVAK, MCCLELLAND, MAIER & NEUSTADT, P.C.
FOURTH FLOOR
1755 JEFFERSON DAVIS HIGHWAY
ARLINGTON, VIRGINIA 22202 U.S.A.

直接電話連絡先：(名前及び電話番号)

Direct Telephone Calls to: (name and telephone number)
(703) 413-3000

単独発明者または第一の共同発明者の氏名	Full name of sole or first joint inventor Kiichiro SAKASHITA	
発明者の署名	日付	Inventor's signature Kiichiro Sakashita Date Oct. 5, 2000
住所	Residence Hyogo, Japan	
国籍	Citizenship Japanese	
郵便の宛先	Post Office Address -c/o KOBE Shipyard & Machinery Works, MITSUBISHI HEAVY INDUSTRIES, LTD. 1-1, Wadasaki-cho, 1-chome, Hyogo-ku, Kobe, Hyogo-ken 652-8585 Japan	
第二の共同発明者の氏名	Full name of second joint inventor, if any Tomohiro ITOH	
第二の共同発明者の署名	日付	Second joint Inventor's signature Tomohiro Ito Date Oct. 5, 2000
住所	Residence Hyogo, Japan	
国籍	Citizenship Japanese	
郵便の宛先	Post Office Address -c/o TAKASAGO Research & Development Center, MITSUBISHI HEAVY INDUSTRIES, LTD. 1-1, Arai-cho Shinhamma 2-chome, Takasago, Hyogo-ken 676-8686 Japan	

(第三以降の共同発明者についても同様に記載し、署名すること)

(Supply similar information and signature for third and subsequent joint inventors.)

Japanese Language Declaration
(日本語宣言書)

第三の共同発明者の氏名	Full name of third joint inventor, if any Katsunari OHSONO
第三の共同発明者の署名 日付	Third joint Inventor's signature Date <i>Katsunari Ohsono</i> Oct. 5, 2000
住所	Residence Hyogo, Japan
国籍	Citizenship Japanese
郵便の宛先	Post Office Address -c/o KOBE Shipyard & Machinery Works, MITSUBISHI HEAVY INDUSTRIES, LTD. 1-1, Wadasaki-cho, 1-chome, Hyogo-ku, Kobe, Hyogo-ken 652-8585 Japan

第四の共同発明者の氏名	Full name of fourth joint inventor, if any Suguru HODE
第四の共同発明者の署名 日付	Fourth joint Inventor's signature Date <i>Suguru Hode</i> Oct. 5, 2000
住所	Residence Hyogo, Japan
国籍	Citizenship Japanese
郵便の宛先	Post Office Address -c/o KOBE Shipyard & Machinery Works, MITSUBISHI HEAVY INDUSTRIES, LTD. 1-1, Wadasaki-cho, 1-chome, Hyogo-ku, Kobe, Hyogo-ken 652-8585 Japan

第五の共同発明者の氏名	Full name of fifth joint inventor, if any
第五の共同発明者の署名 日付	Fifth joint Inventor's signature Date
住所	Residence
国籍	Citizenship
郵便の宛先	Post Office Address

第六の共同発明者の氏名	Full name of sixth joint inventor, if any
第六の共同発明者の署名 日付	Sixth joint Inventor's signature Date
住所	Residence
国籍	Citizenship
郵便の宛先	Post Office Address

(第六またはそれ以降の共同発明者に対しても同様な情報および署名を提供すること。)

(Supply similar information and signature for third and subsequent joint inventors.)